
Neutron induced reaction measurements in accelerator facilities

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Introduction

motivation (Nuclear Data and Applications)

Principles and experimental details

Neutron x-section and Time Of Flight concept facilities (GELINA, n_TOF)

measurement setups

Data reduction

analysis process

neutron flux – capture yield

applications (archaeology, research, safeguards etc.)

Summary

future perspectives/links

Nuclear waste management:

a major pressing and potentially costly environmental problem

Sources:

- Power generation reactors, military (e.g. dismantled weapons), other applications (medicine, industry, research)

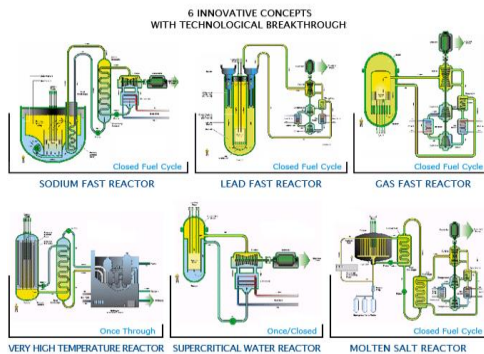
Treatment:

- Surface storage or geological disposal

Need to improve specific nuclear data



Produce x-sections/improve uncertainty



Waste disposal issues



- Where (?) - geological and public issues
- How (?) – technological aspects
- Long term safety (studies on effects projected for the next 10^5 y!)

Possible solution:
Transmutation

Burn or transform a long-lived isotope to a short-lived or a stable one via neutron induced fission or capture

**Advanced reactor concepts:
Generation IV reactors and Accelerator Driven Systems (ADS)**

- Main challenge: to be able to recycle large fraction of the long-lived radioactive waste during operation
- R&D on several fields required

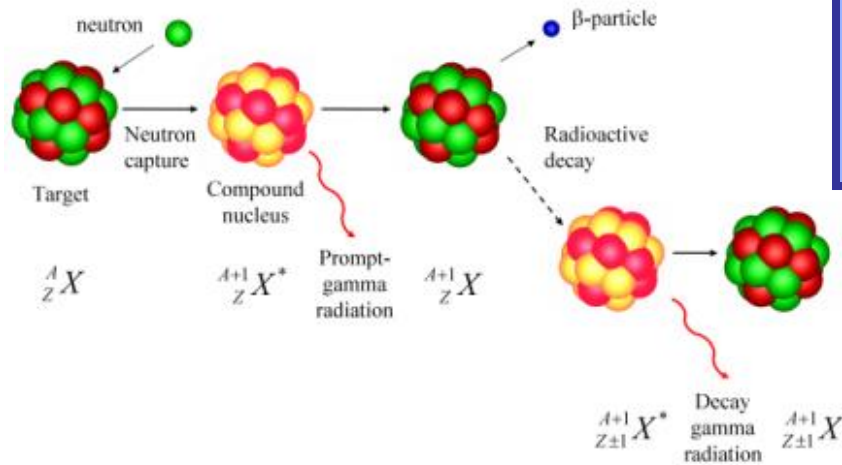


**NEED
for new and more accurate neutron data**

uncertainties

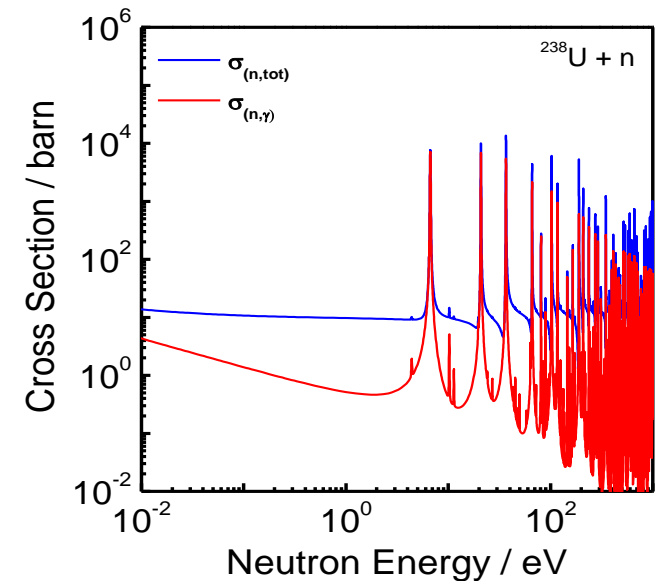
Nuclear reactions induced by neutrons

- elastic scattering, inelastic scattering, **capture (n,γ)**, fission (n,f), (n,xn), (n,a)...etc.

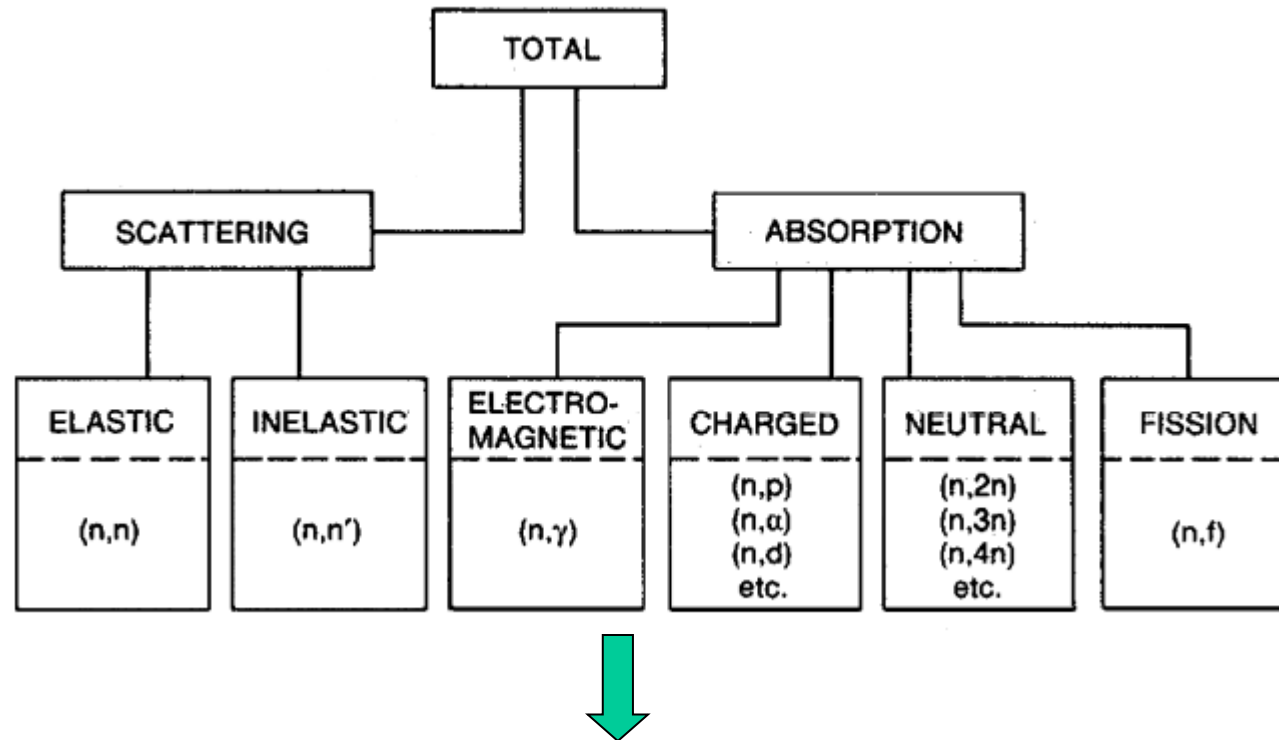


The probability that a specific interaction will take place between an incident neutron and a nucleus is expressed by a quantity called **cross section, σ**

The cross section depends strongly on the velocity of the neutron (E_n) and interacting nucleus



Neutron induced reactions



For every type of reaction we can determine the corresponding cross section: σ_{tot} , σ_f , σ_γ , σ_{el}

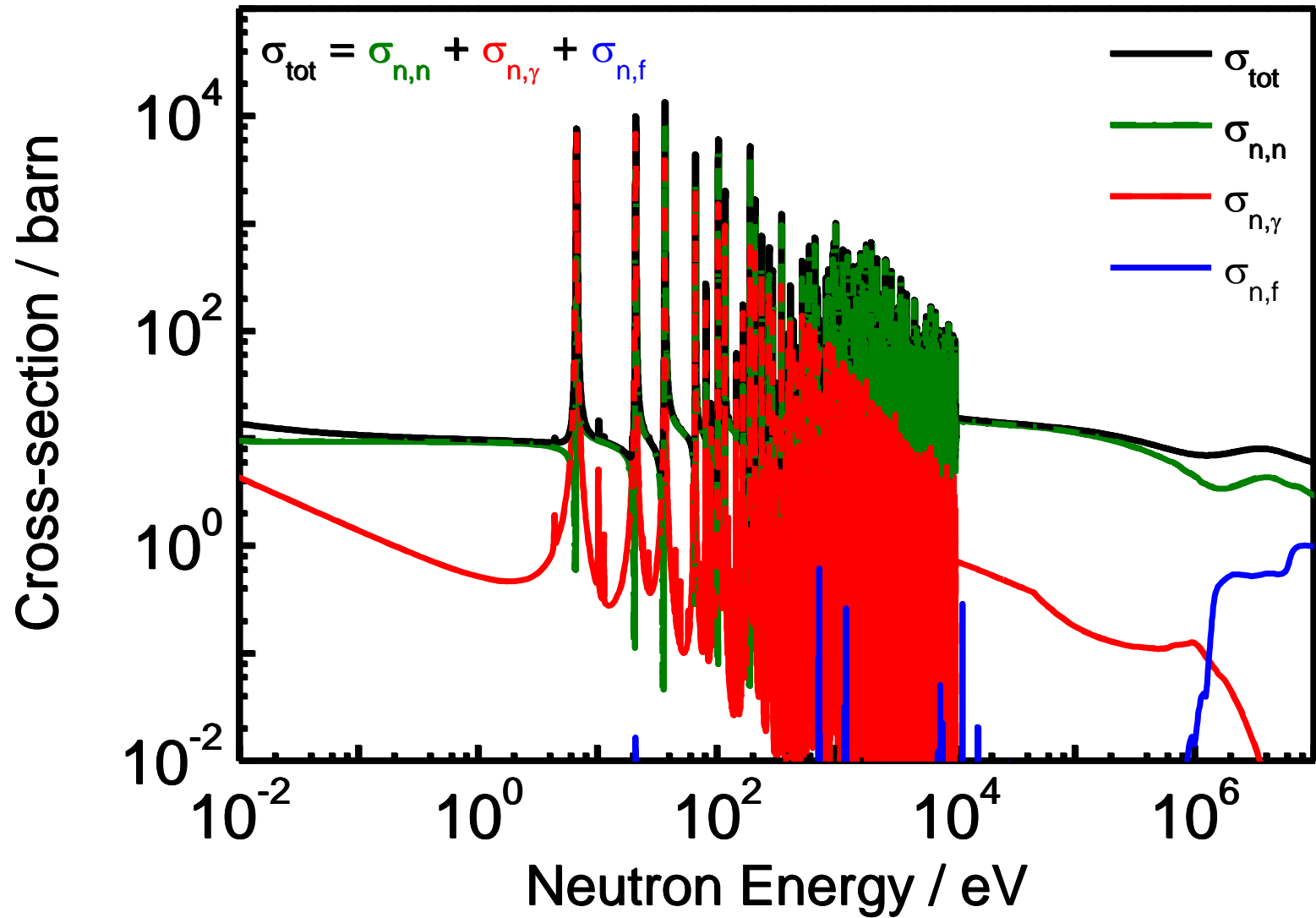
Neutron cross section

irfu

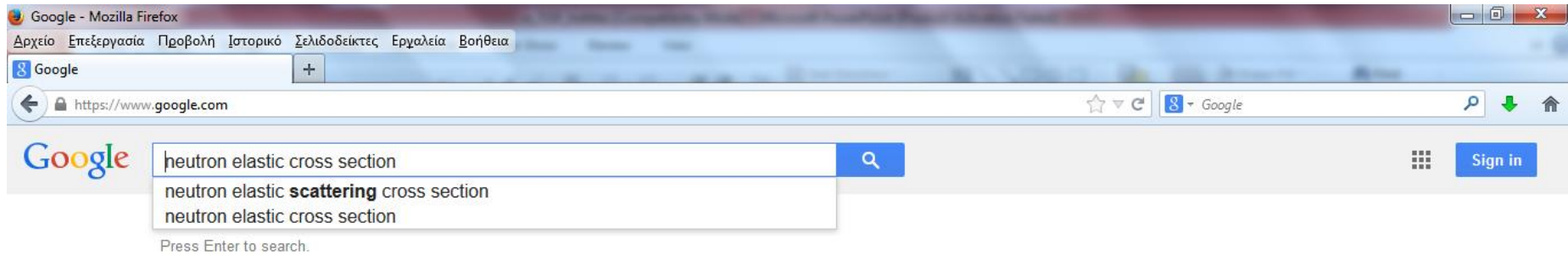
cea

saclay

$^{238}\text{U} + n$



Quest for a specific number



?

Nuclear data

search for a specific value and you end up



experimental results

evaluated files

EXFOR

- www-nds.iaea.org/exfor/
- www.nndc.bnl.gov/exfor/
- www.jcprg.org/exfor/
-
-

ENDF

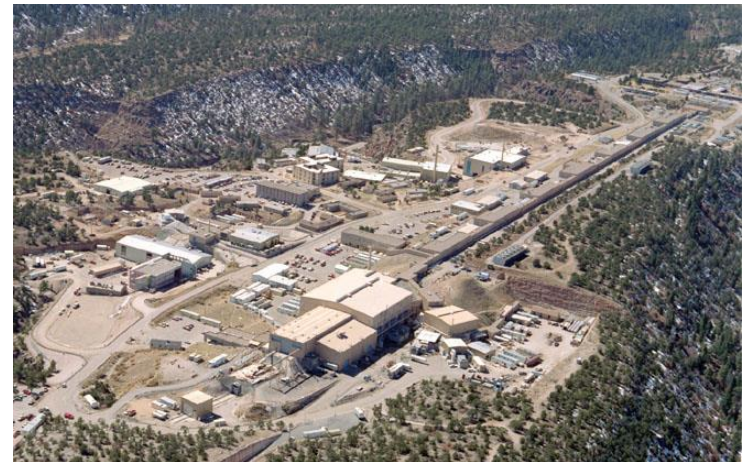
- *BROND (Russia)*
- *CENDL (China)*
- *ENDF (USA)*
- *JEFF (international)*
- *JENDL (Japan)*

An evaluated nuclear data library is a data file (of specific format) of recommended nuclear data

Neutron induced reactions are under study since 1950s

- in numerous facilities
- for the vast majority of isotopes

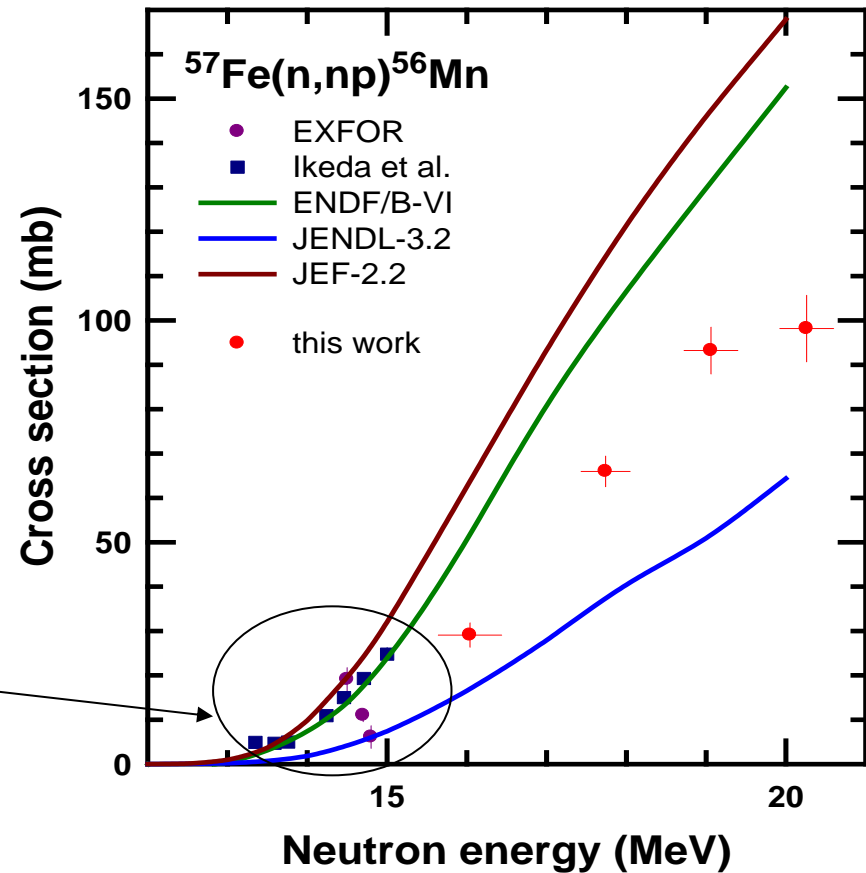
- **LOS ALAMOS**
- **OAK RIDGE**
- **JAPAN**
- **RUSSIA**
- **n_TOF (CERN)**
- **GELINA (IRMM)**
- **VdG accelerators (France, Germany, Belgium, Greece etc.)**
- **HIGH-FLUX REACTORS**



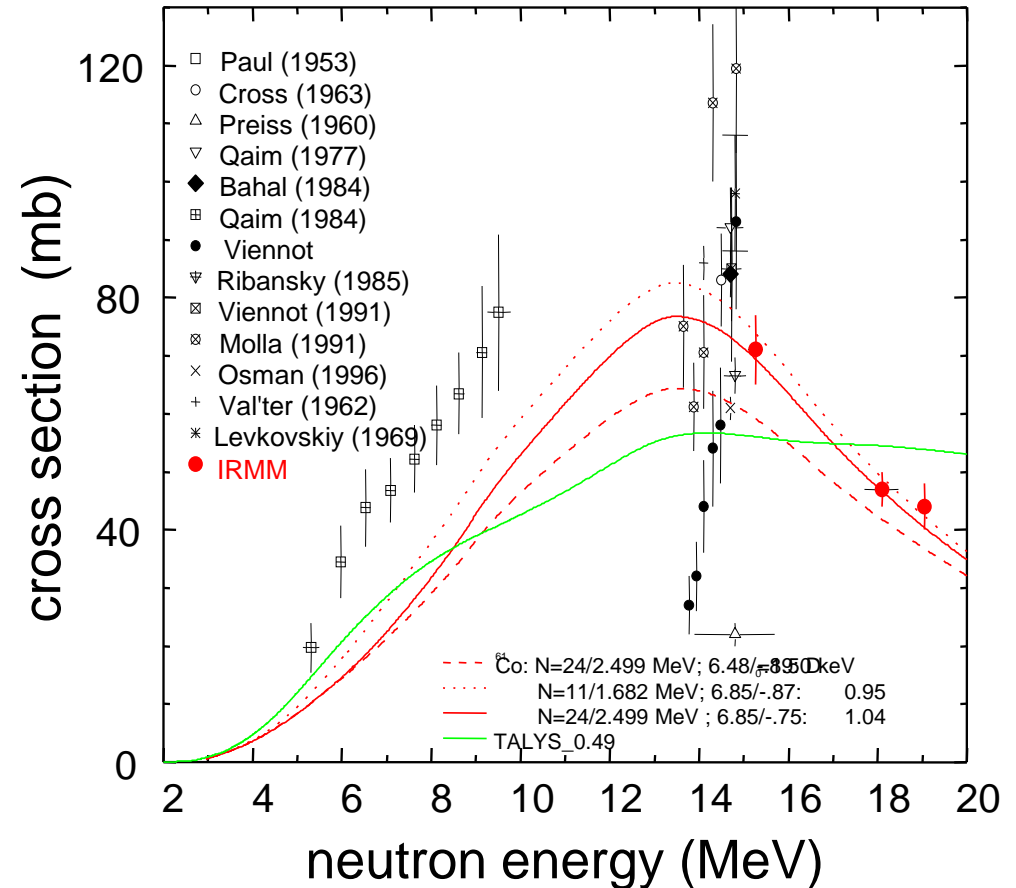
Why continue experiments ?

large uncertainties in data files due to **lack** of experimental data

previous knowledge

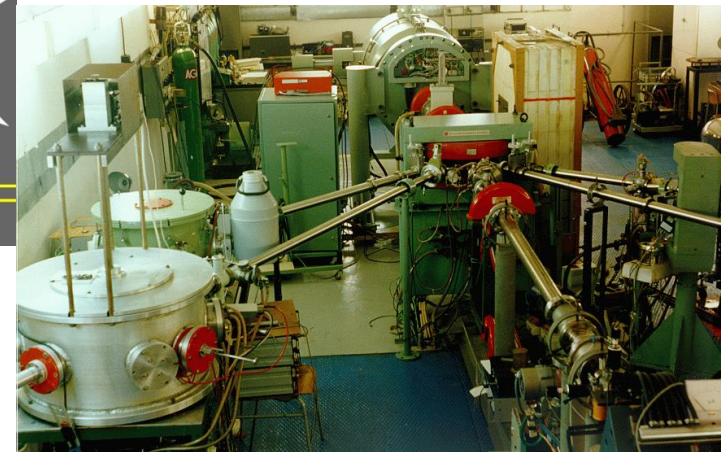
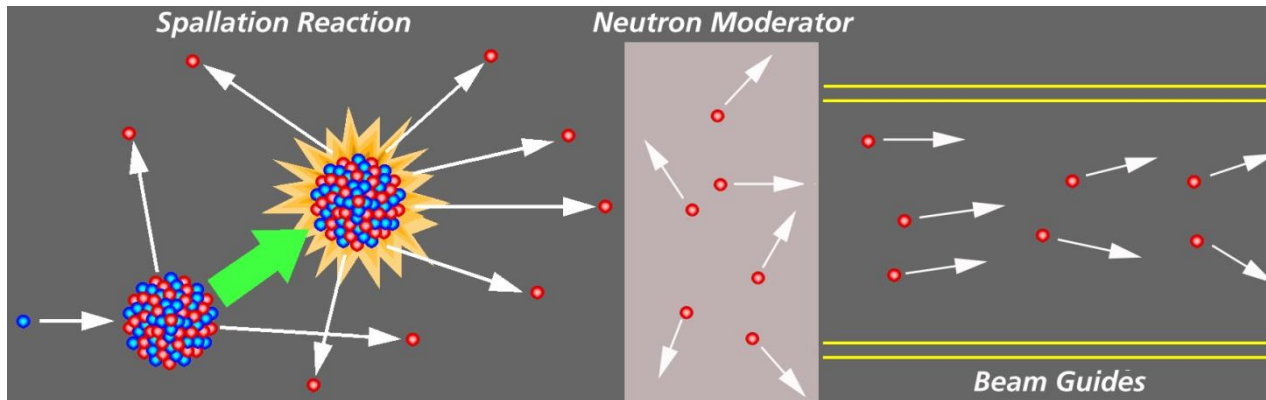


large uncertainties in data files due to **inconsistency** of experimental data



Neutron beams are mainly produced:

- In reactors via **fission**
- In accelerators via the **spallation** process after bombarding heavy elements with energetic particles like a proton
- In accelerators via **charged particle reactions** and subsequent neutron production



Separate according to neutron spectrum

monochromatic

pulsed 'white' source

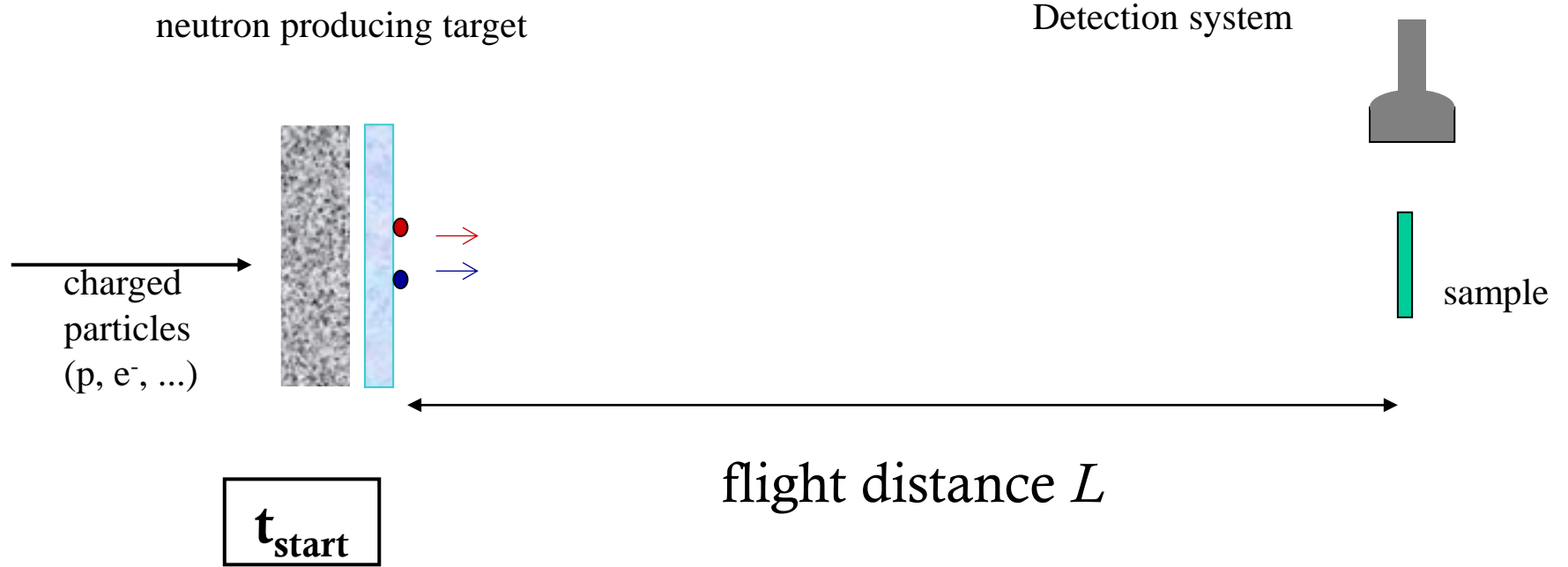


Athens-Greece

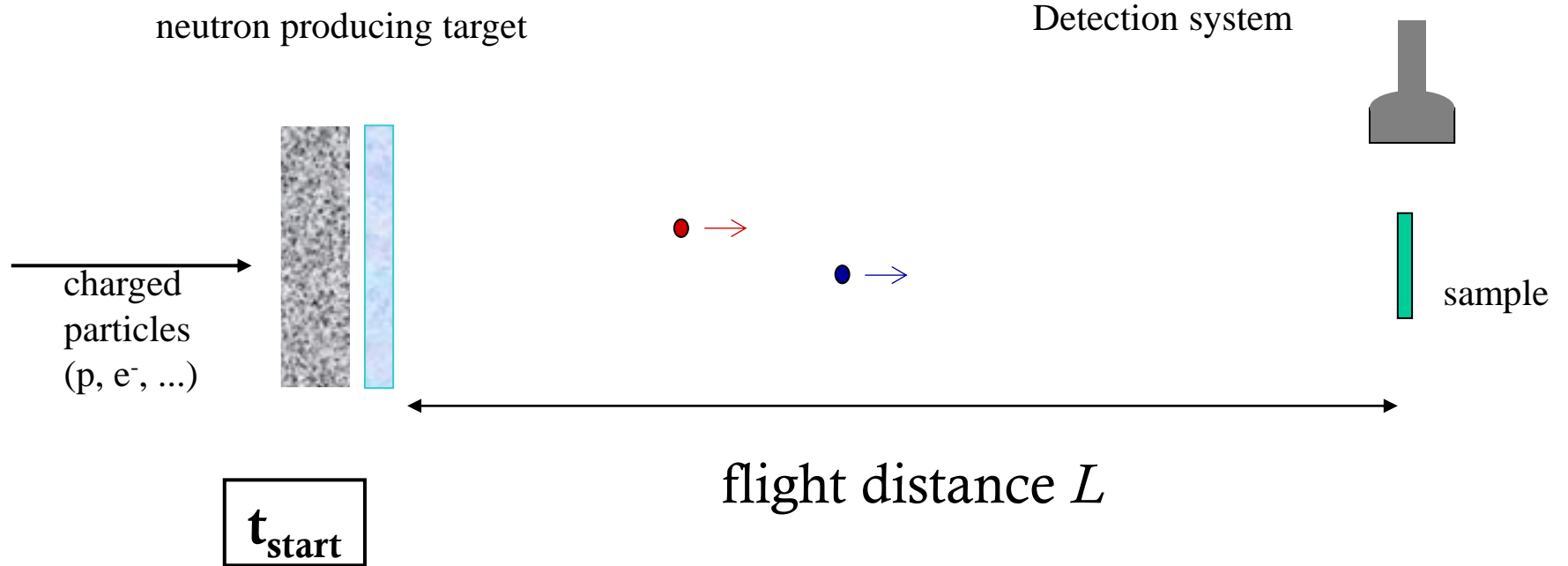


Geel-Belgium

the Time Of Flight principle

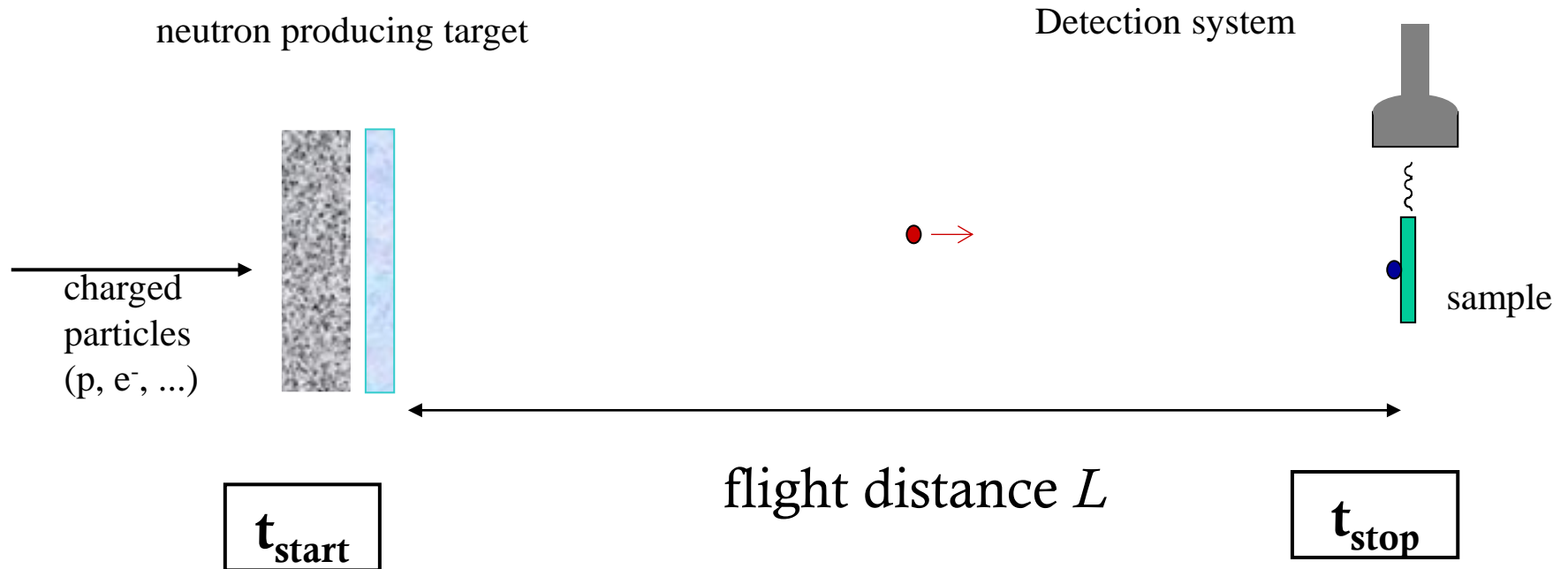


Time Of Flight



Time Of Flight

We have to measure two quantities: the distance and time required by the particle to reach the end of the given flight



$$\text{ToF} = t_{\text{stop}} - t_{\text{start}}$$

$$E_n = \frac{1}{2} m v^2 = \frac{1}{2} m L^2 / (\text{ToF})^2$$

Neutron racing

irfu

cea

saclay



t_{start}

t



Neutron racing

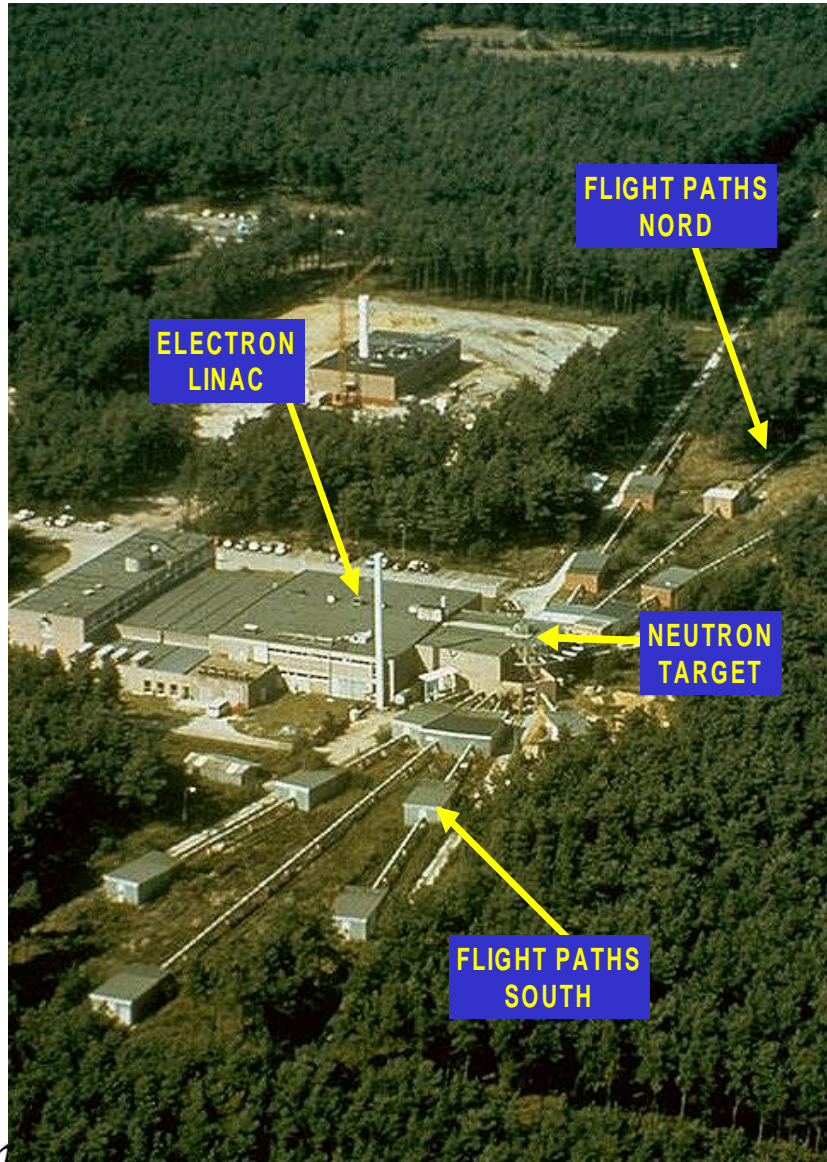


Flight path length ?!

t_{stop}



GELINA facility (JRC-IRMM)



Time-Of-Flight facility

Pulsed white neutron source

$$(10 \text{ meV} < E_n < 20 \text{ MeV})$$

Multi-user facility with 10 flight paths

$$(10 \text{ m} - 400 \text{ m})$$

The measurement stations have equipment to perform:

Total cross section measurements

Partial cross section measurements

Pulse width : 1 ns

Frequency : 50 – 800 Hz

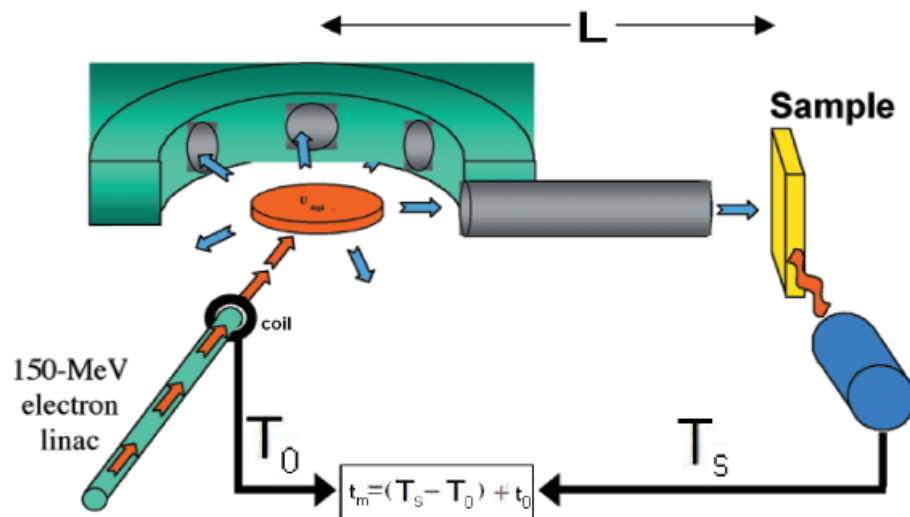
Neutron intensity : $1.6 \cdot 10^{12} - 2.5 \cdot 10^{13}$ n/s

We need to determine the velocity of the neutron

$$E_n = \frac{1}{2}(m_n v_n^2) \text{ where } v_n = L/t *$$

so we have to measure **L** and **t**

* non-relativistic

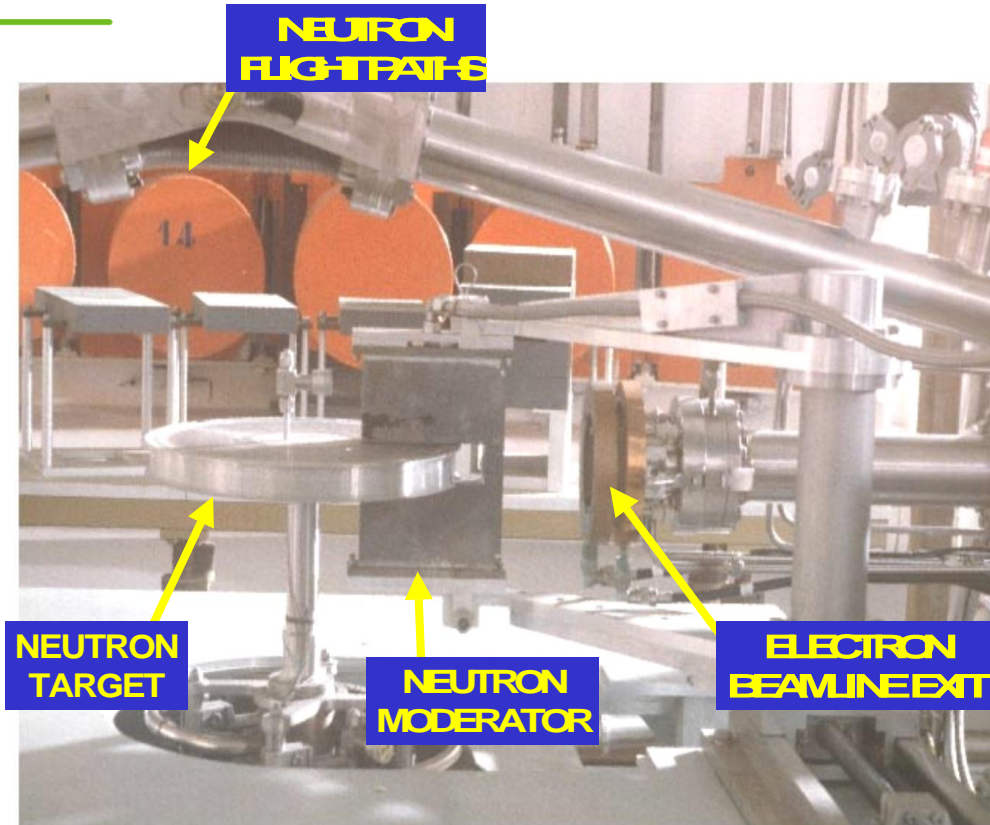


L: the distance neutrons 'fly'
t: the TOF equals to $T_s - T_0$

GELINA neutron source

irfu

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- e^- accelerated to $E_{e^-, \max} \approx 140$ MeV
- (e^-, γ) Bremsstrahlung in U-target (rotating & cooled with liquid Hg)
- (γ, n) , (γ, f) in U-target
- Low energy neutrons by water moderator in Be-canning

GELINA control room



GELINA control room - 1965



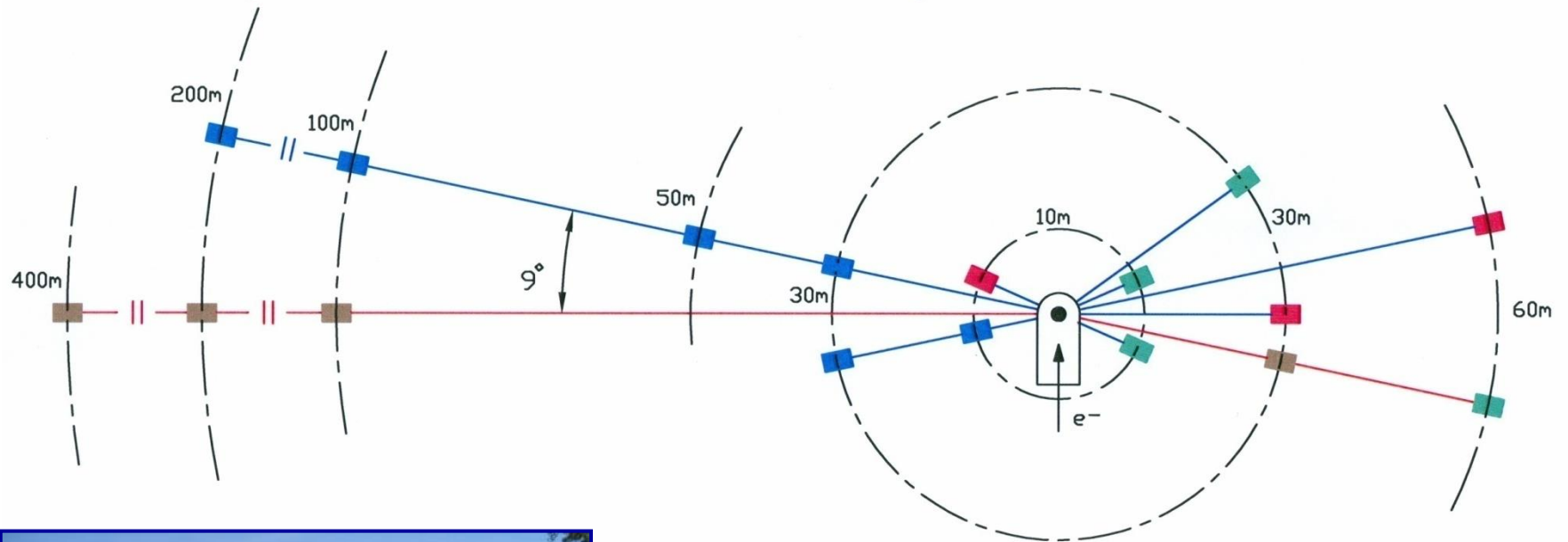
GELINA control room - 2009





GELINA top view

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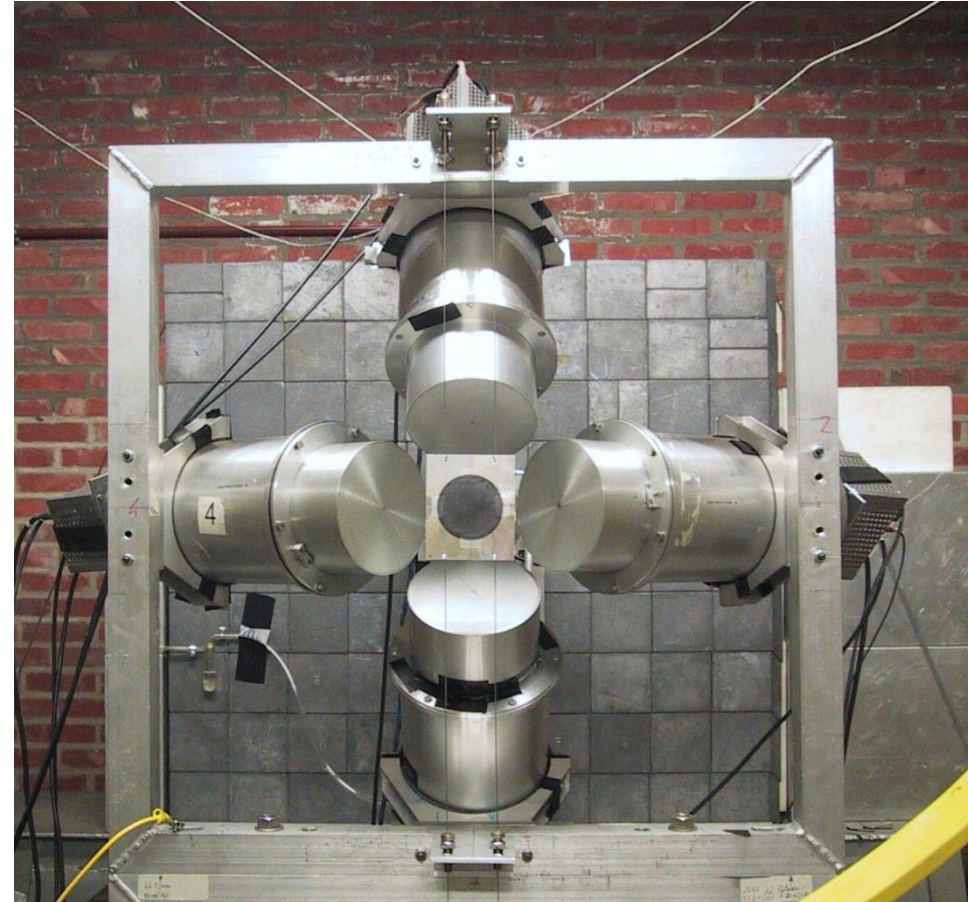


	(n,γ)	NIM, A577 (2007) 626
	(n,tot)	NP A 773, 173 (2006)
	(n,f) and (n,cp)	NSE 156, 211 (2007)
	$(n,n'\gamma)$	NP A 786, 1 (2007)

L = 12.9 m

Total energy detection

- C_6D_6 liquid scintillators
 - 125°
 - PHWT
- Flux measurements (IC)
 - $^{10}B(n,\alpha)$

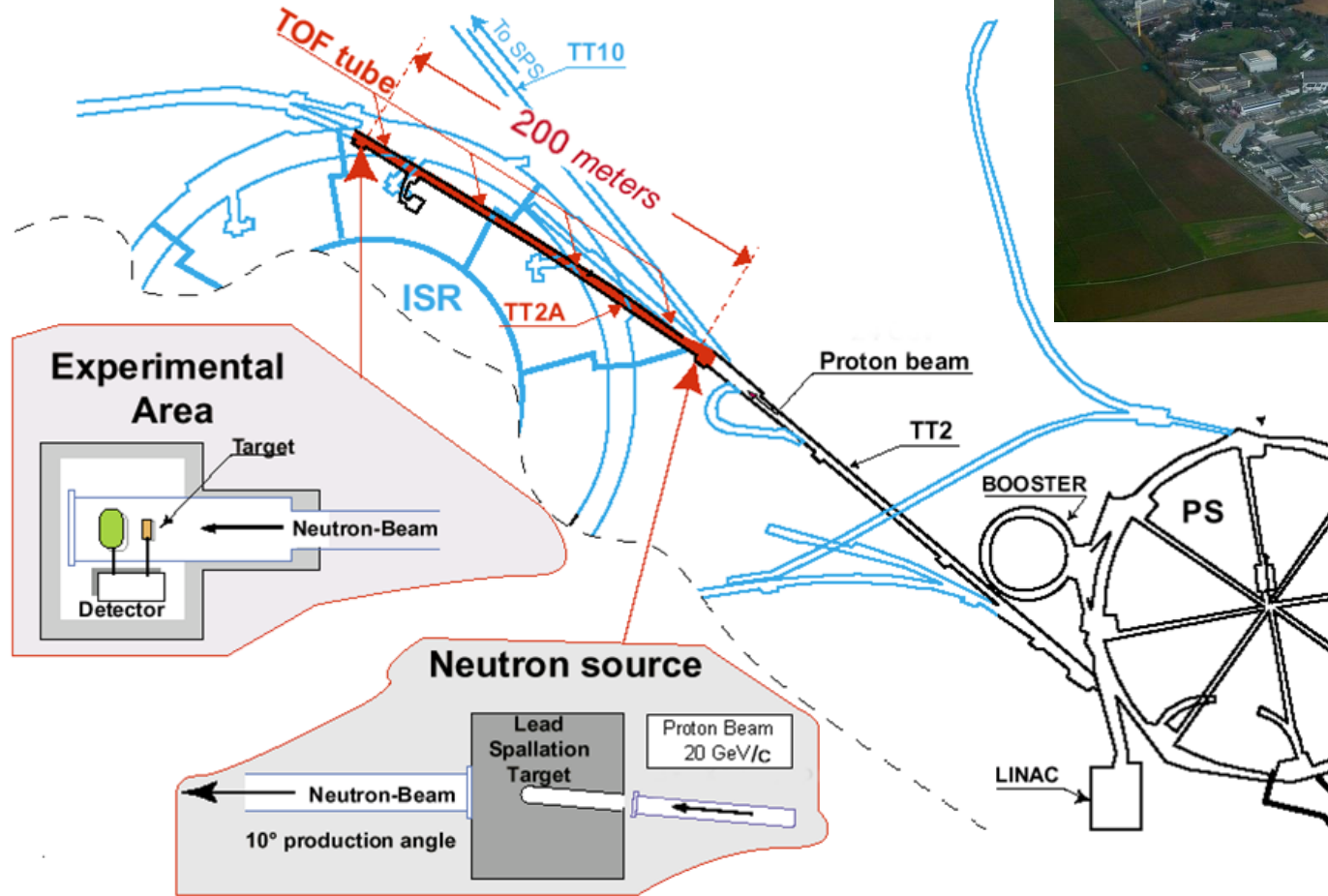


n_TOF facility (CERN)

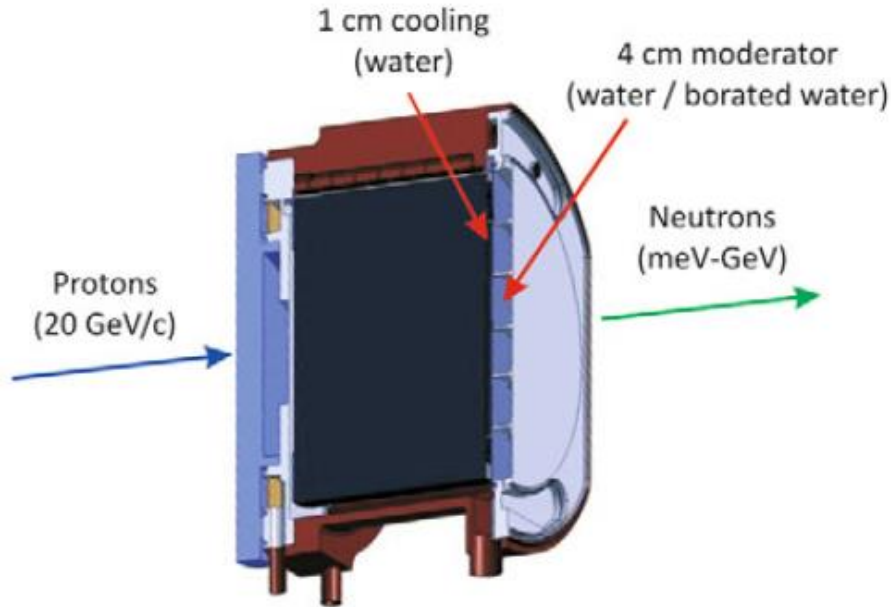
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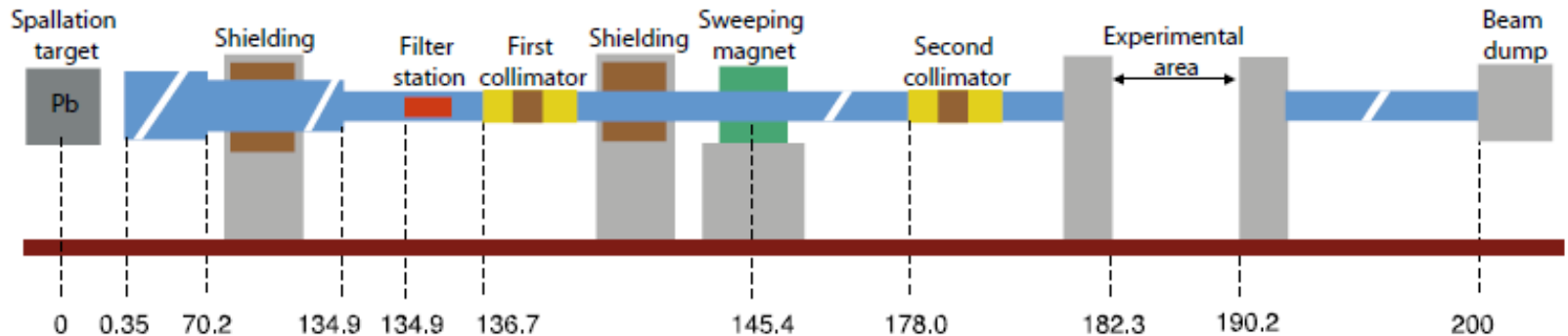
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n_TOF neutron source



- accelerated protons of 20 GeV/c hit the lead block target
- 7×10^{12} neutrons ppp
- partial cross section measurements setups
- 200m flight path length

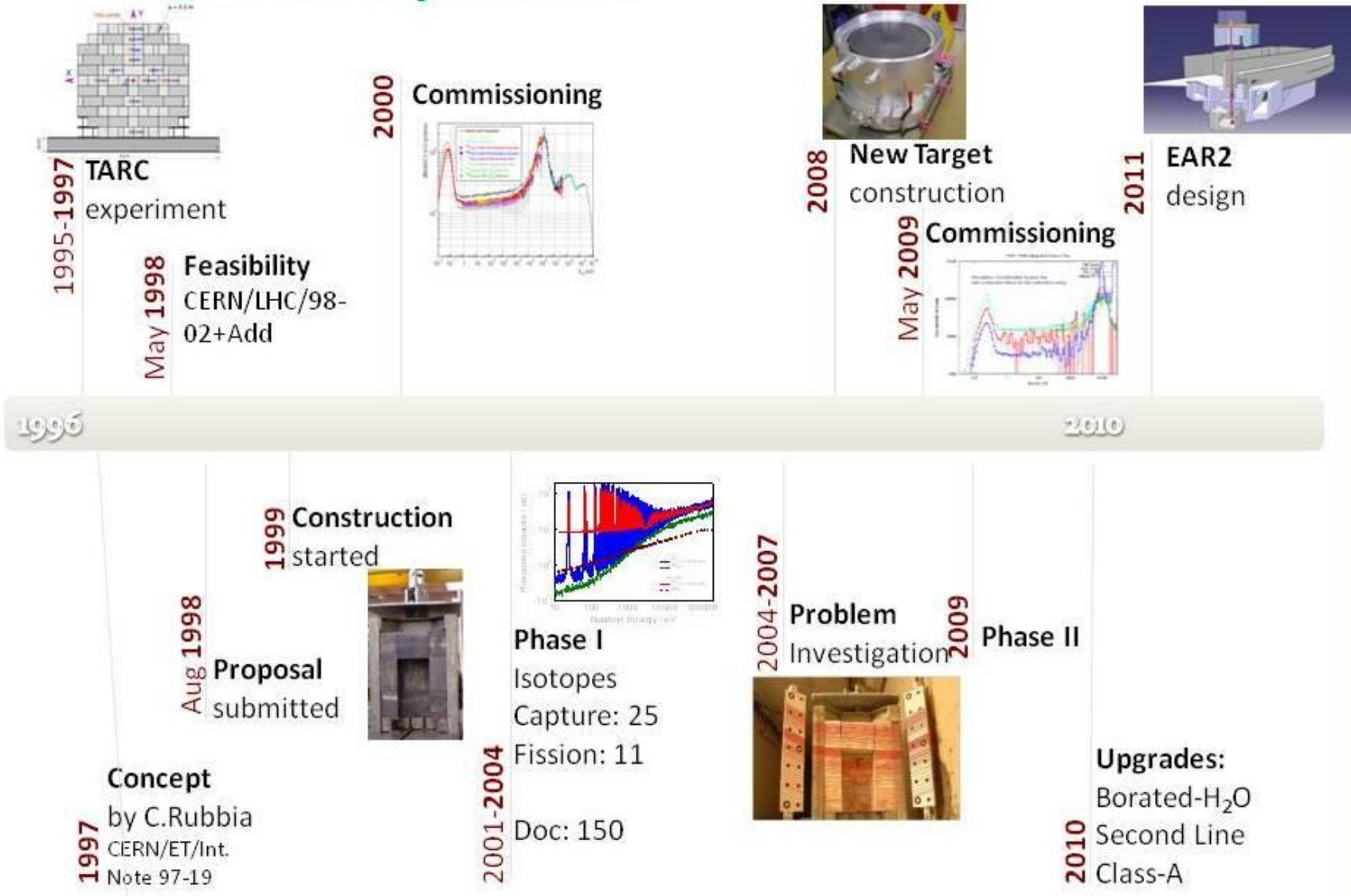


n_TOF tunnel



n_TOF facility history

n_TOF Facility Timeline



Capture (n, γ)

^{151}Sm

$^{204,206,207,208}\text{Pb}$

^{209}Bi

^{232}Th

$^{24,25,26}\text{Mg}$

$^{90,91,92,94,96}\text{Zr}$

^{93}Zr

^{139}La

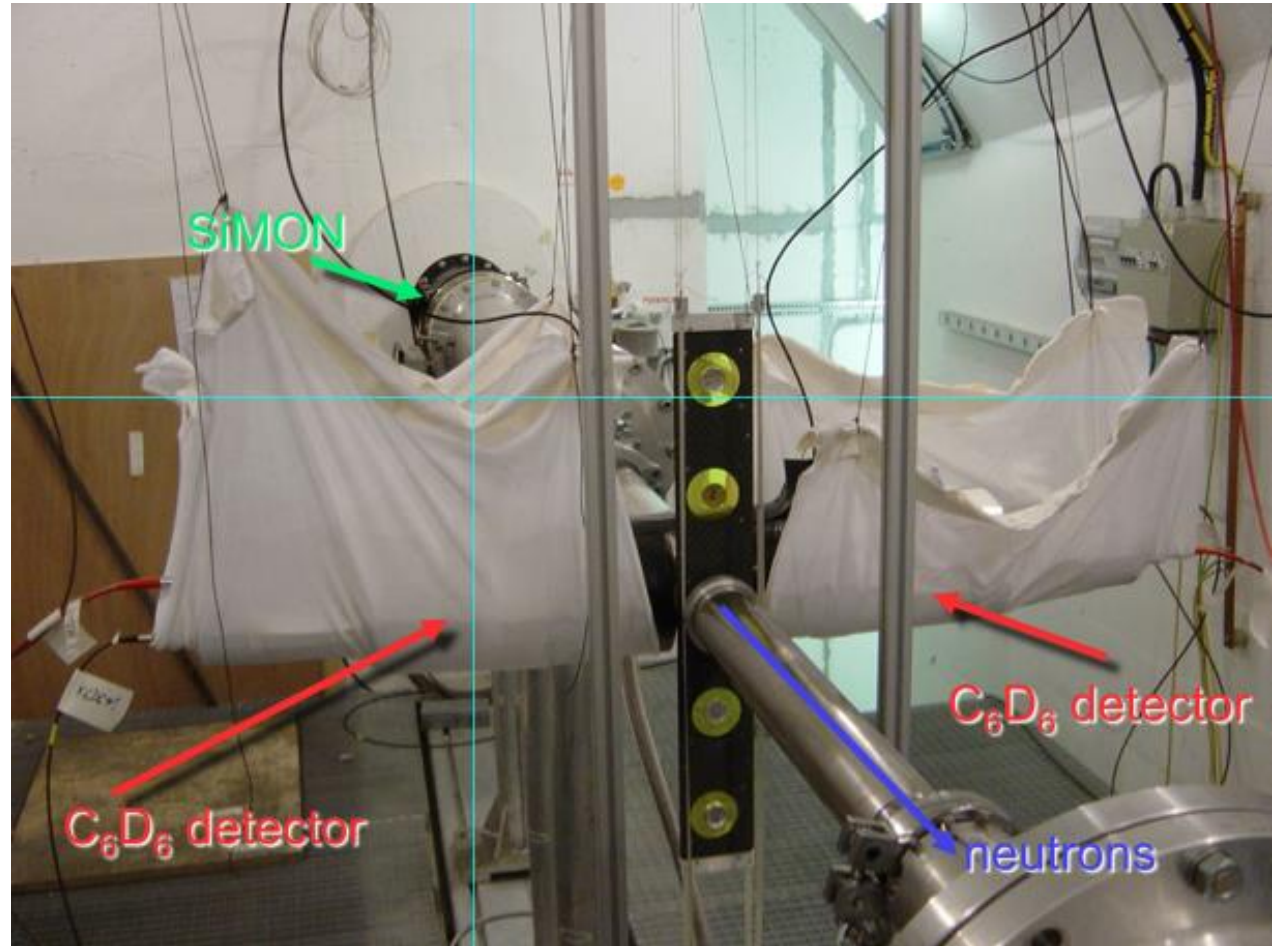
$^{186,187,188}\text{Os}$

^{197}Au

$^{233,234}\text{U}$

$^{237}\text{Np}, ^{240}\text{Pu}$

^{243}Am



Capture setup (n_TOF)

Capture (n, γ)

^{151}Sm

$^{204,206,207,208}\text{Pb}$

^{209}Bi

^{232}Th

$^{24,25,26}\text{Mg}$

$^{90,91,92,94,96}\text{Zr}$

^{93}Zr

^{139}La

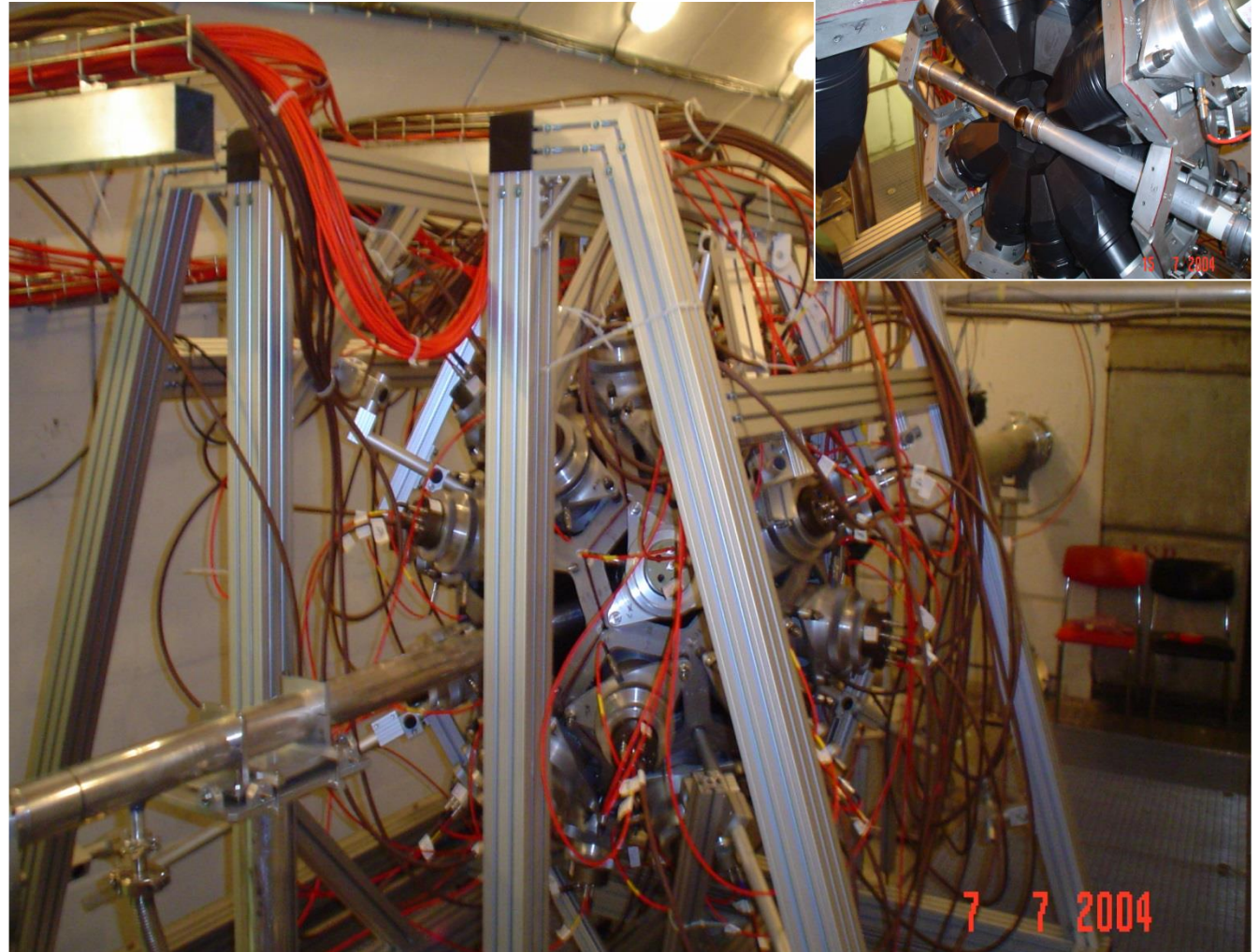
$^{186,187,188}\text{Os}$

^{197}Au

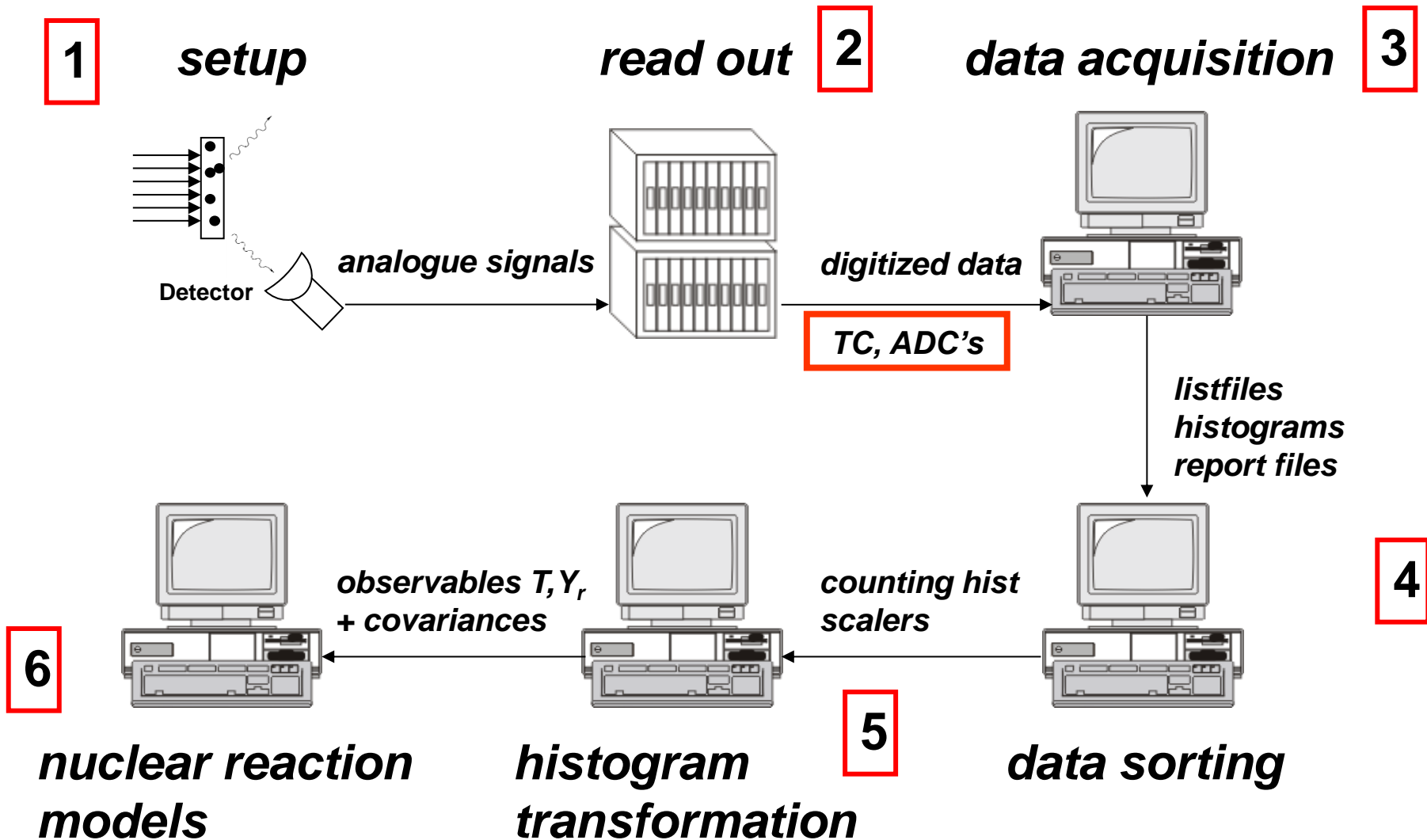
$^{233,234}\text{U}$

$^{237}\text{Np}, ^{240}\text{Pu}$

^{243}Am



Data reduction workflow

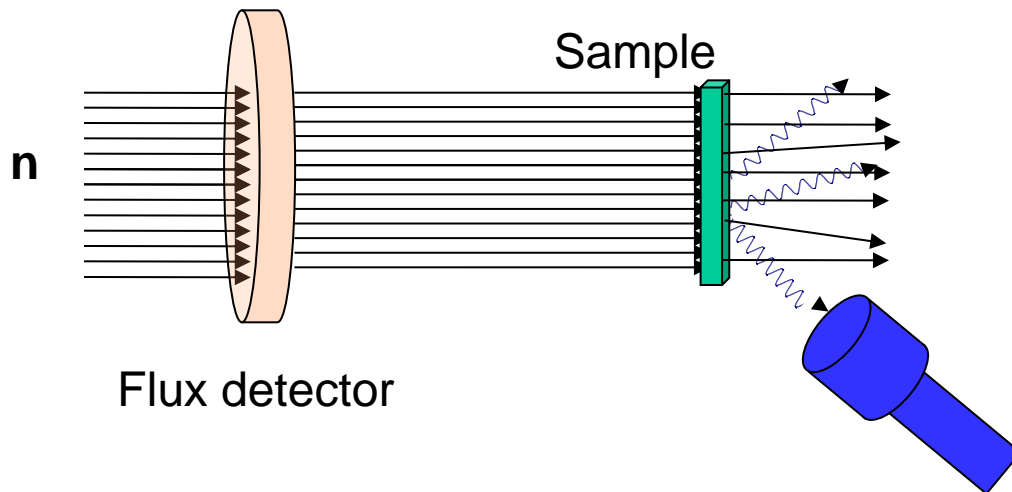


Basic principles

Determined quantity: capture yield, Y_c

Results via fitting code: REFIT

$$Y_{\text{exp}} = N \frac{C'_w - B'_w}{C'_\phi - B'_\phi} Y_\phi$$



Need to determine accurately:
Net neutron FLUX count rate
Net CAPTURE count rate

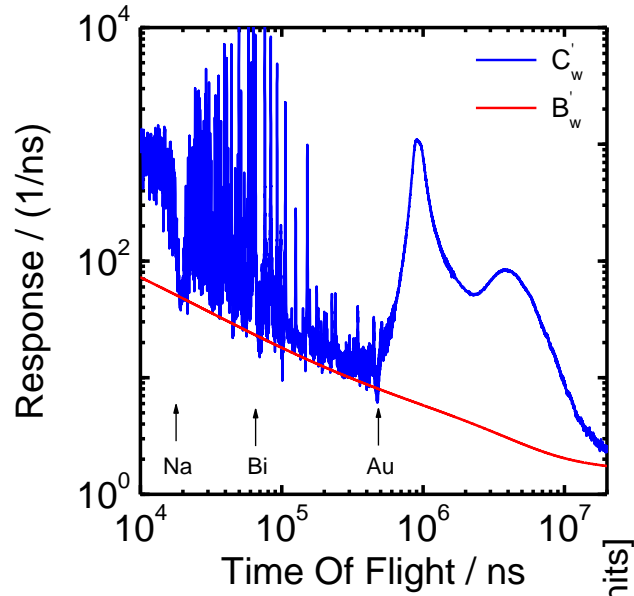
Measured quantities

irfu

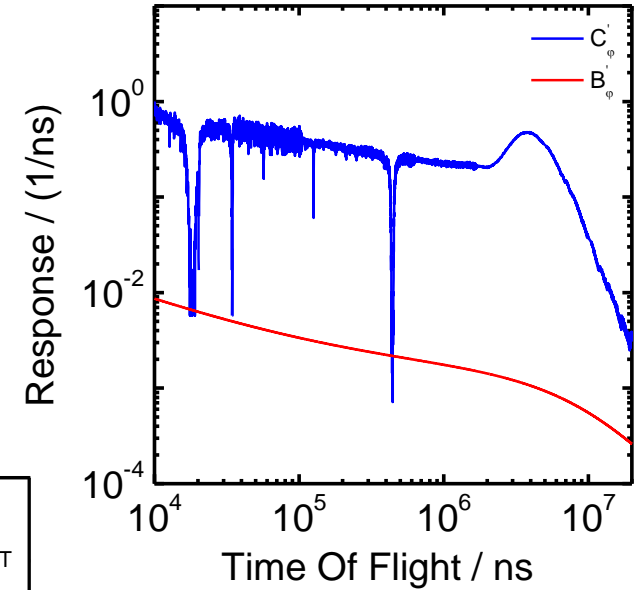
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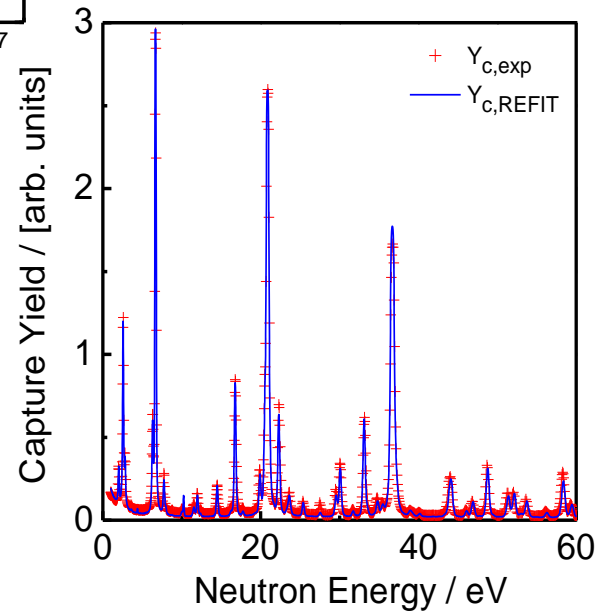
Capture



Flux



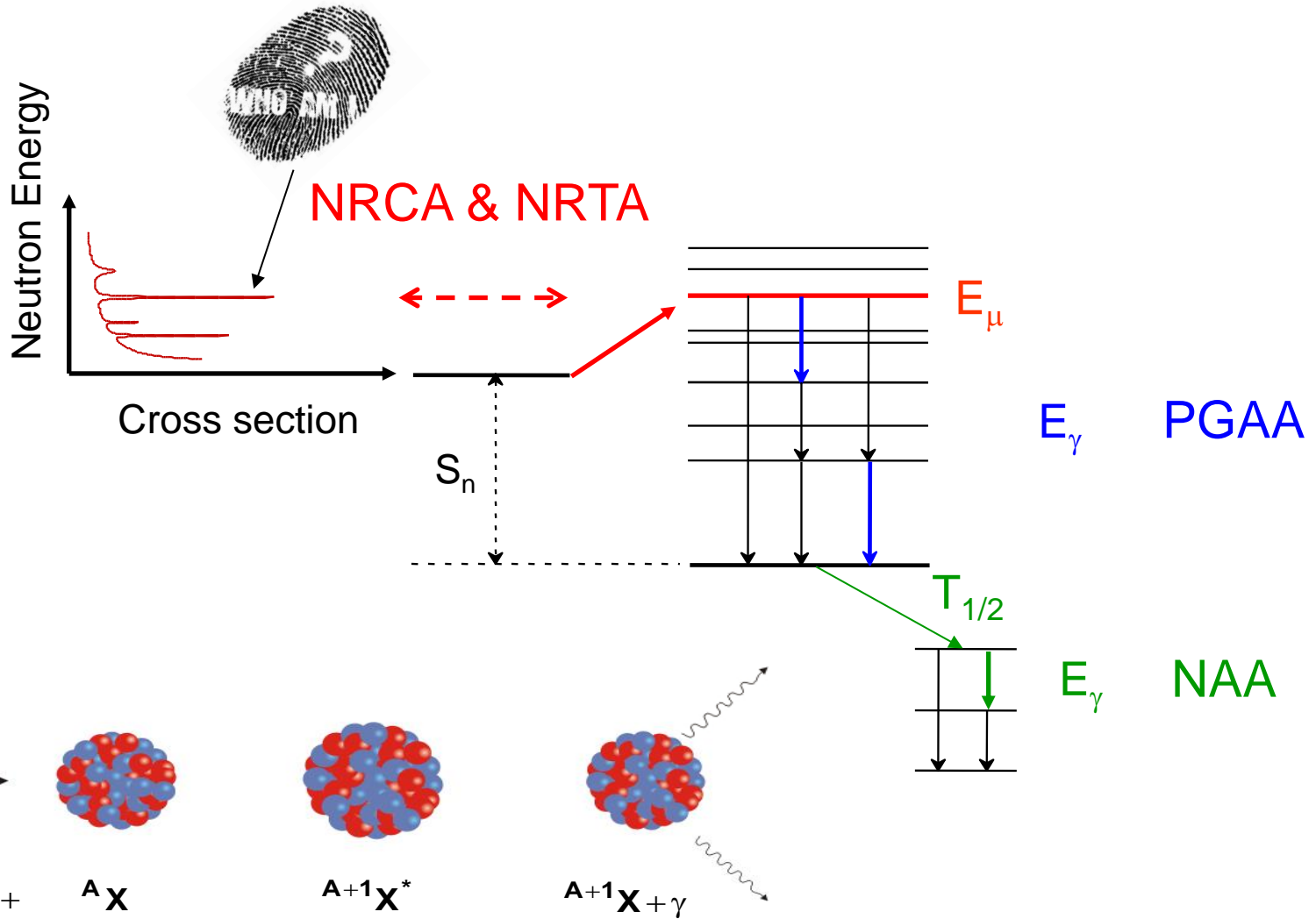
Yield



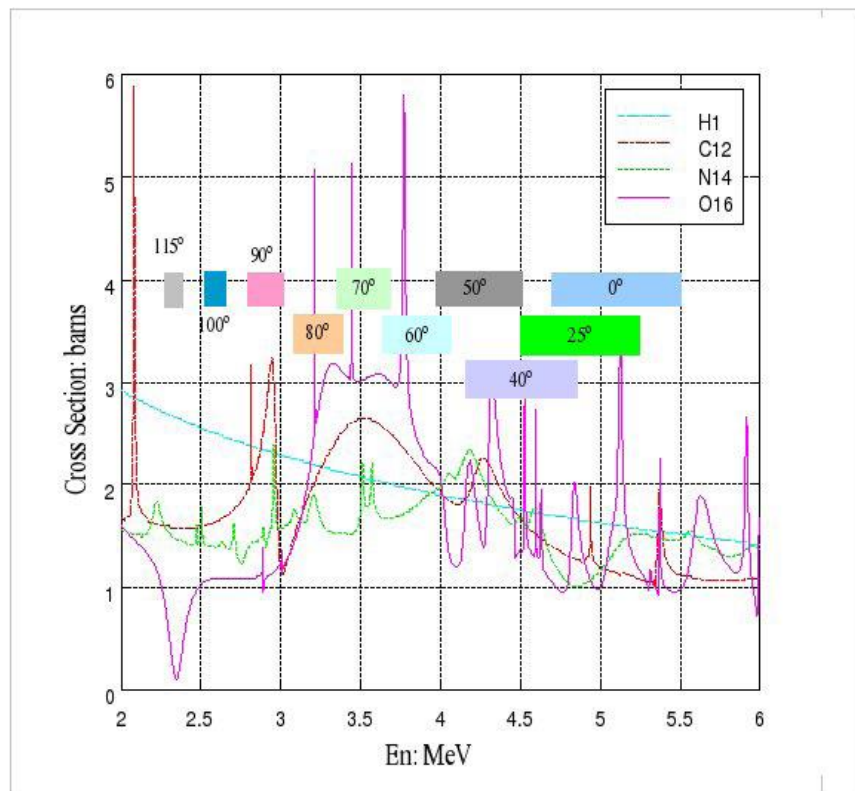
Many applications make use of neutrons within different fields

- ✓ Neutrons in the service of Art (Archaeology)
- ✓ Security and safety (Detection of illegal substances and explosives)
- ✓ Research and Industry (Elemental analysis and isotopic composition)
- ✓ Industry and economy (localization of diamonds in rocks)

Resonances used as fingerprints



Detection of explosives (monoenergetic)



Neutron source

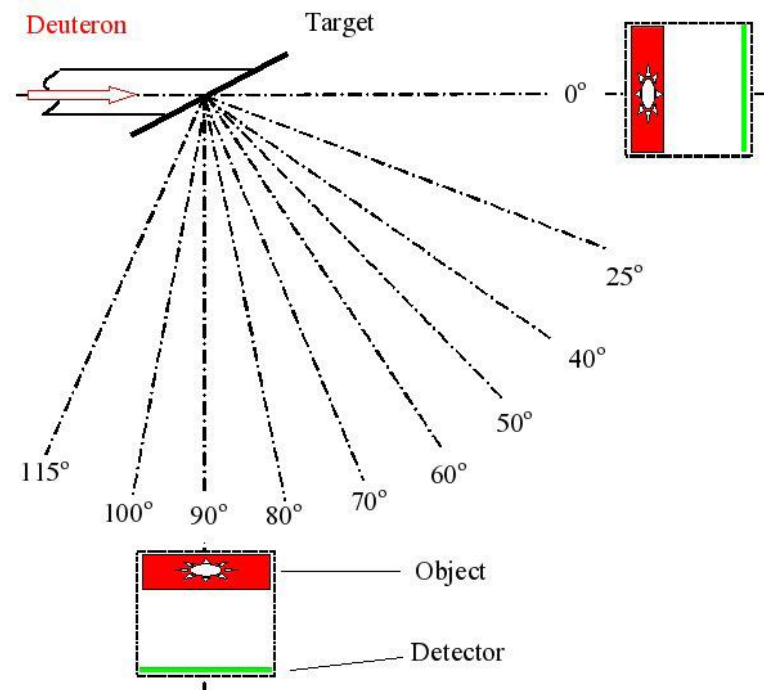
- $d(d,n)^3\text{He}$ at $E_d = 2.5$ MeV

Neutron detector

- a plastic scintillator viewed by a CCD camera

Different angles

- solve a set of linear equations for n_H , n_C , n_N and n_O



G. Chen and R.C. Lanza, IEEE Transactions on Nuclear Science, 49 (2002) 1919 – 1924

Always have alternatives



Pros and Cons

- ✓ These are full non-destructive methods especially compared to other methods
- ✓ No special preparation of the samples required
- ✓ Reliable results based on well established procedures

- ✗ Require special facilities or devices
- ✗ Place restrictions (public places)
- ✗ Specialized personnel and time consuming analysis

Neutron experiments and related studies

- ✓ Always room for improvements in experimental performance and analysis techniques
- ✓ Neutron data needs arise from demands (energy, safety, industry...)
- ✓ Research slowed down recently – budget cuts (Fukushima effect)
- ✓ Many new applications using neutrons as probes

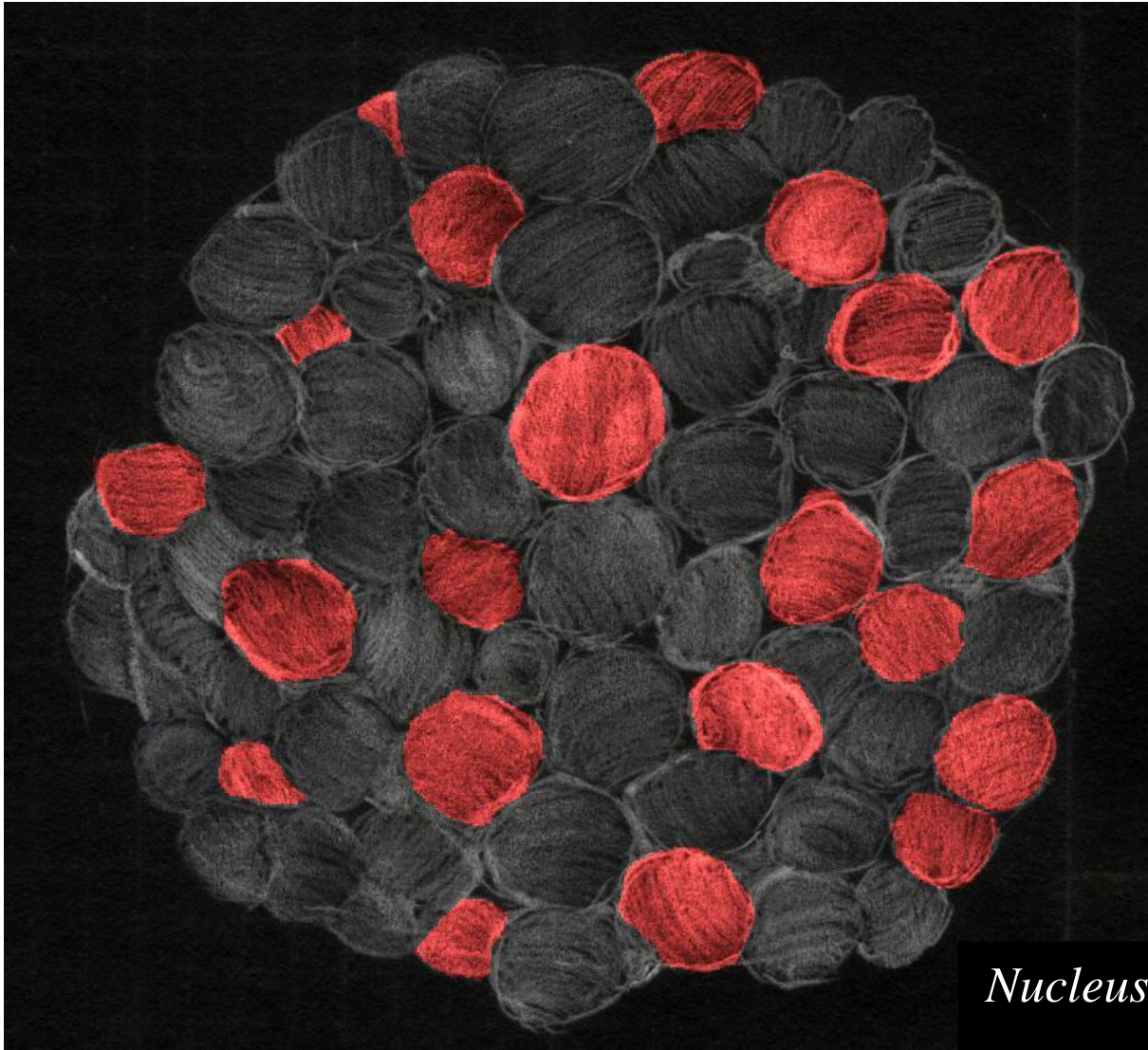
Need for new scientists



You can find additional information

- ✓ irmm.jrc.ec.europa.eu/
- ✓ cern.ch/ntof
- ✓ lansce.lanl.gov/
- ✓ neutrons.ornl.gov/
- ✓ <http://www.neutron.anl.gov/facilities.html>

Thank you



*Nucleus of U-235: protons in red,
neutrons in grey.
(Art by Blake Stacey.)*